



# Pebble Database for PBR MC&A – NEUP

Advanced Reactor Safeguards Spring Working Group Meeting, October 31 – November 2, 2023

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# VCU

College of Engineering

Mechanical and Nuclear Engineering

# Project Team

## Core VCU team

- Braden Goddard (PI)
- Zeyun Wu (co-PI)
- Zachary Crouch (Ph.D. student)
- **Kashminder Mehta (Ph.D. student)**
- Ben Impson (undergrad)

- Project duration: Oct. 2022 – Sept. 2024
- Funding amount: \$400k

## External advisory team

- Claudio Gariazzo (ANL)
- Yonggang Cui (BNL)
- Philip Gibbs (ORNL)
- Donny Hartanto (ORNL)

# The Challenge

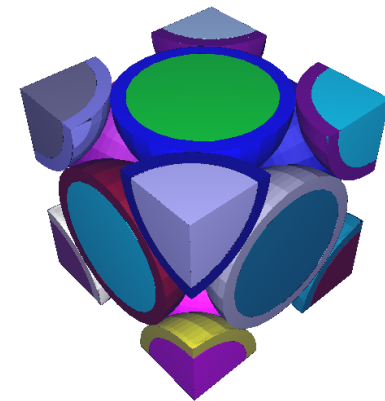
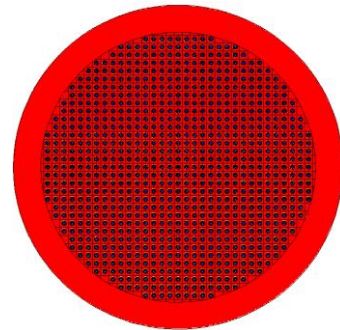
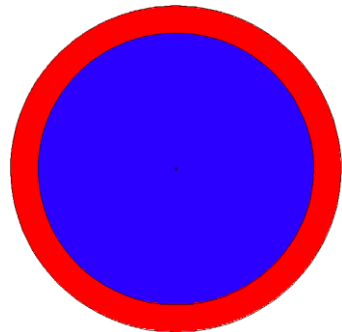
- Knowing the **nuclear and radiological material content** in used pebbles is important for:
  - Safeguards
  - Facility operations
  - Waste management
  - Etc.
- Used pebbles are **measured relatively quickly after discharge** and their **path through the reactor** can vary between pebbles
  - Traditional LWR gamma and neutron NDA correlations may not be applicable

# Project Goal

- Create a data library of used pebble NDA signatures
  - Gamma spectra (HPGe)
  - Neutron counts
- Validate data library using an independent code
  - INDEPTH (ORIGEN)
  - OpenMC
- Document methodology used to create the data library
  - Focus is Xe-100

# Previous Work

- Initial work as focused on:
  - Recruiting and training students
  - Creating pebble variations to understand modeling sensitivities
    - Reflective boundary mirror vs white
    - Homogeneous vs heterogeneous
    - Simple cubic vs face centered cubic
    - Latticed TRISO particles vs semi-random
    - Effect of clipped TRISO particles
    - Reflective boundary at pebble surface vs cube with helium



# Comparison of “Random” Cases

- There are different methods to simulate the randomness seen in particle distributions
- It is believed that the method employed by OpenMC most closely matches the real distribution of particles

Model	$k_{\infty}$
Uniform	$1.50820 \pm 0.00007$
URAN	$1.50929 \pm 0.00006$
Semi-Random	$1.51203 \pm 0.00008$
Random-OpenMC	$1.51071 \pm 0.00012$

# Conclusions from INMM 2023 Presentation

- All TRISO fuel is the same
  - Excluding enrichment
- Fuel is manufactured as specified in the nuclear industry
- 1% perturbations will have an ~300 pcm or less change in fresh fuel
- Changes that effect moderation appear to have large impacts on nuclear material content in spent fuel
  - More analyses is needed
- TRISO particle distribution is difficult to model perfectly

# Code Validation MCNP-OpenMC

Single fresh pebble at room temperature (293.62 K)

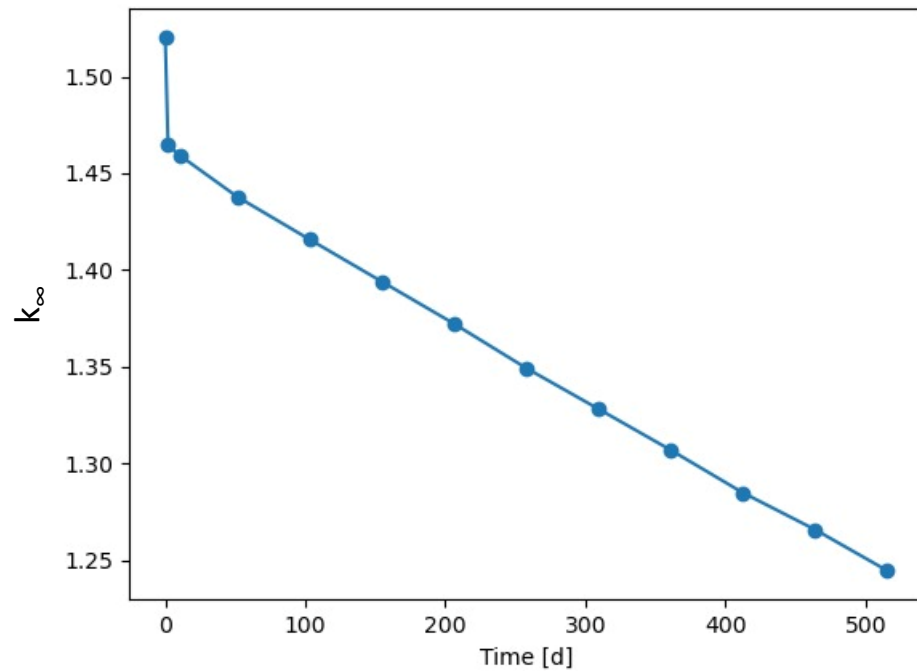
Pebble Model		$k_{\infty}$ (White B.C.)	$k_{\infty}$ (Mirror B.C.)	Diff. in $k_{\infty}$
Uniform	MCNP	$1.60743 \pm 0.00008$	$1.61471 \pm 0.00004$	-0.00728
	OpenMC	$1.60818 \pm 0.00011$	$1.61560 \pm 0.00012$	-0.00742
	dev. in $k_{\infty}$	$-67 \times 10^{-5}$	$-89 \times 10^{-5}$	
URAN	MCNP	$1.60830 \pm 0.00005$	$1.61563 \pm 0.00006$	-0.00733
Semi-Random		$1.61017 \pm 0.00007$	$1.61723 \pm 0.00006$	-0.00706
Random-OpenMC	OpenMC	$1.61025 \pm 0.00011$	$1.61739 \pm 0.00012$	-0.00714
Compare to URAN	dev. in $k_{\infty}$	$187 \times 10^{-5}$	$176 \times 10^{-5}$	
Compare to Semi-Random	dev. in $k_{\infty}$	$8 \times 10^{-5}$	$16 \times 10^{-5}$	

Single fresh pebble at hot temperature (1200 K)

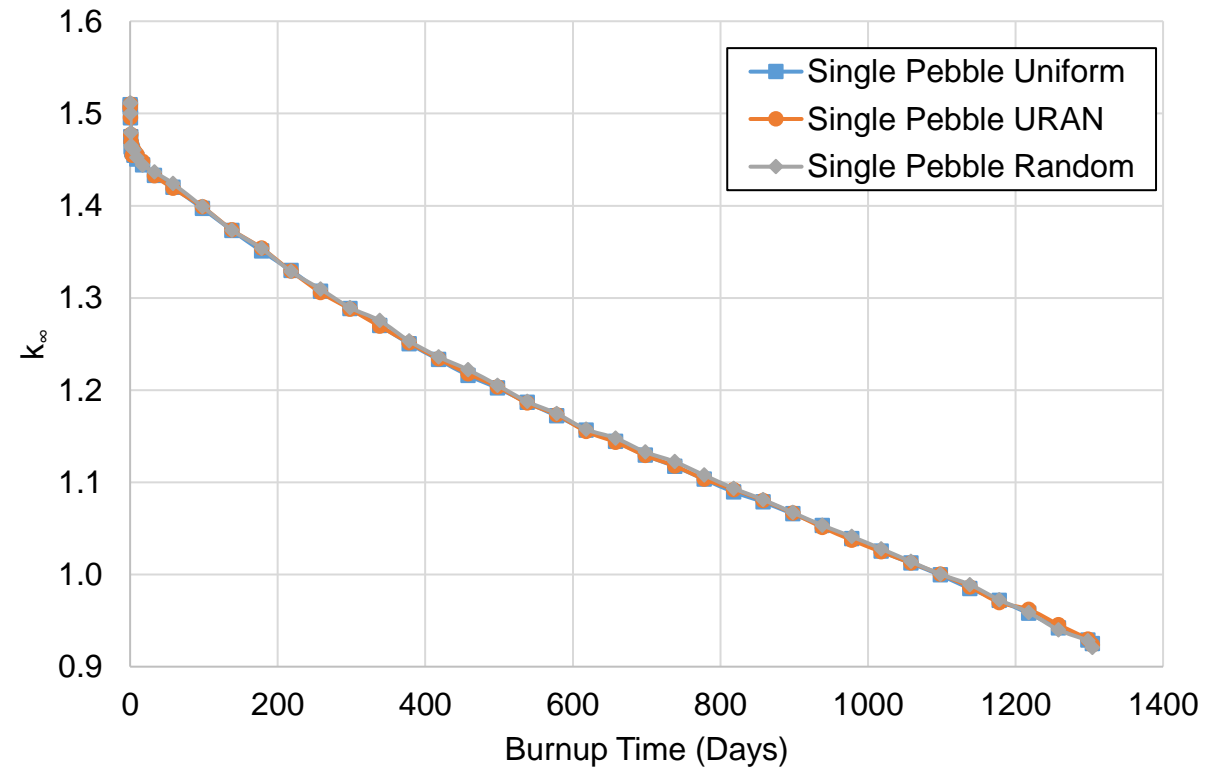
Pebble Model		$k_{\infty}$ (White B.C.)	$k_{\infty}$ (Mirror B.C.)	Diff. in $k_{\infty}$
Uniform	MCNP	$1.50820 \pm 0.00007$	$1.51774 \pm 0.00007$	-0.00954
	OpenMC	$1.50789 \pm 0.00012$	$1.51757 \pm 0.00012$	-0.00968
	dev. in $k_{\infty}$	$31 \times 10^{-5}$	$17 \times 10^{-5}$	
URAN	MCNP	$1.50929 \pm 0.00006$	$1.51866 \pm 0.00004$	-0.00937
Semi-Random		$1.51203 \pm 0.00008$	$1.52111 \pm 0.00006$	-0.00908
Random-OpenMC	OpenMC	$1.51071 \pm 0.00012$	$1.51980 \pm 0.00012$	-0.00909
Compare to URAN	dev. in $k_{\infty}$	$142 \times 10^{-5}$	$114 \times 10^{-5}$	
Compare to Semi-Random	dev. in $k_{\infty}$	$-132 \times 10^{-5}$	$-131 \times 10^{-5}$	



# Burnup: $k_{\infty}$

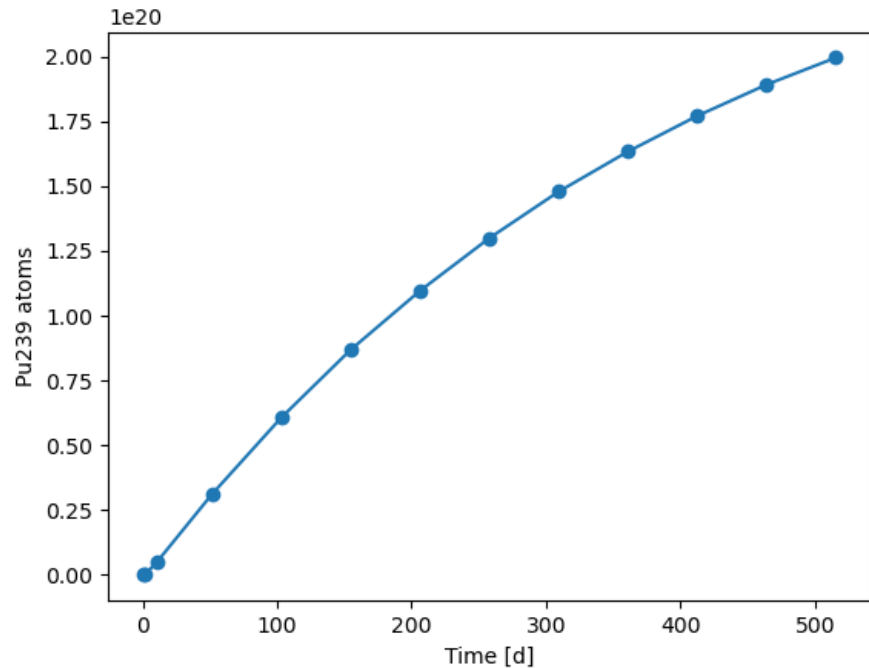


Burnup: OpenMC  $k_{\infty}$  mirror B.C. model

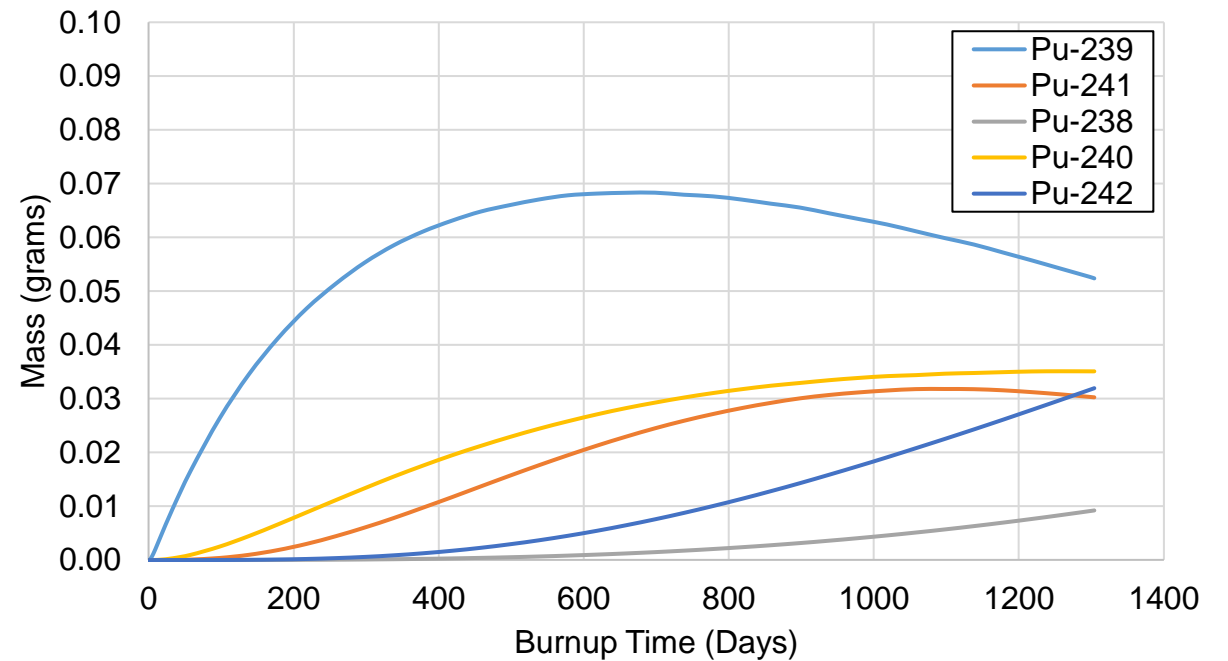


Burnup: MCNP  $k_{\infty}$  white B.C. model

# Burnup: Plutonium Isotopic Content



Burnup: OpenMC Pu content mirror B.C. model



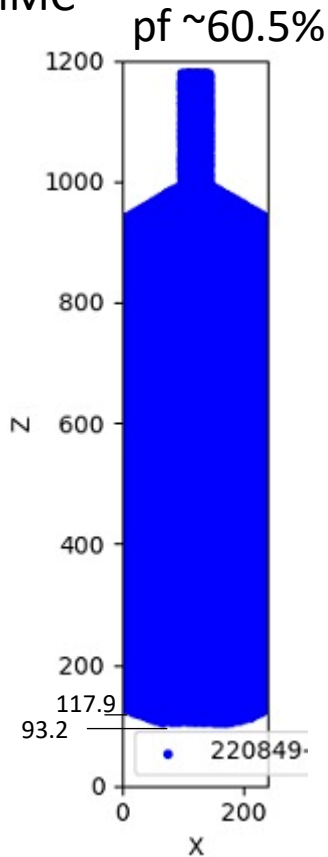
Burnup: MCNP Pu content white B.C. URAN model

# INDEPTH Validation

- Feed MCNP nuclide data into INDEPTH
  - INDEPTH library needed slight modification
    - Thanks Brandon Grogan for help
- Xe-100 reactor ORIGEN model is needed
  - Thanks Jonathan Wing for sharing your model!
- Will compare to AGR, MAGNOX, and PWR fuel models
  - INDEPTH results should not match for these models

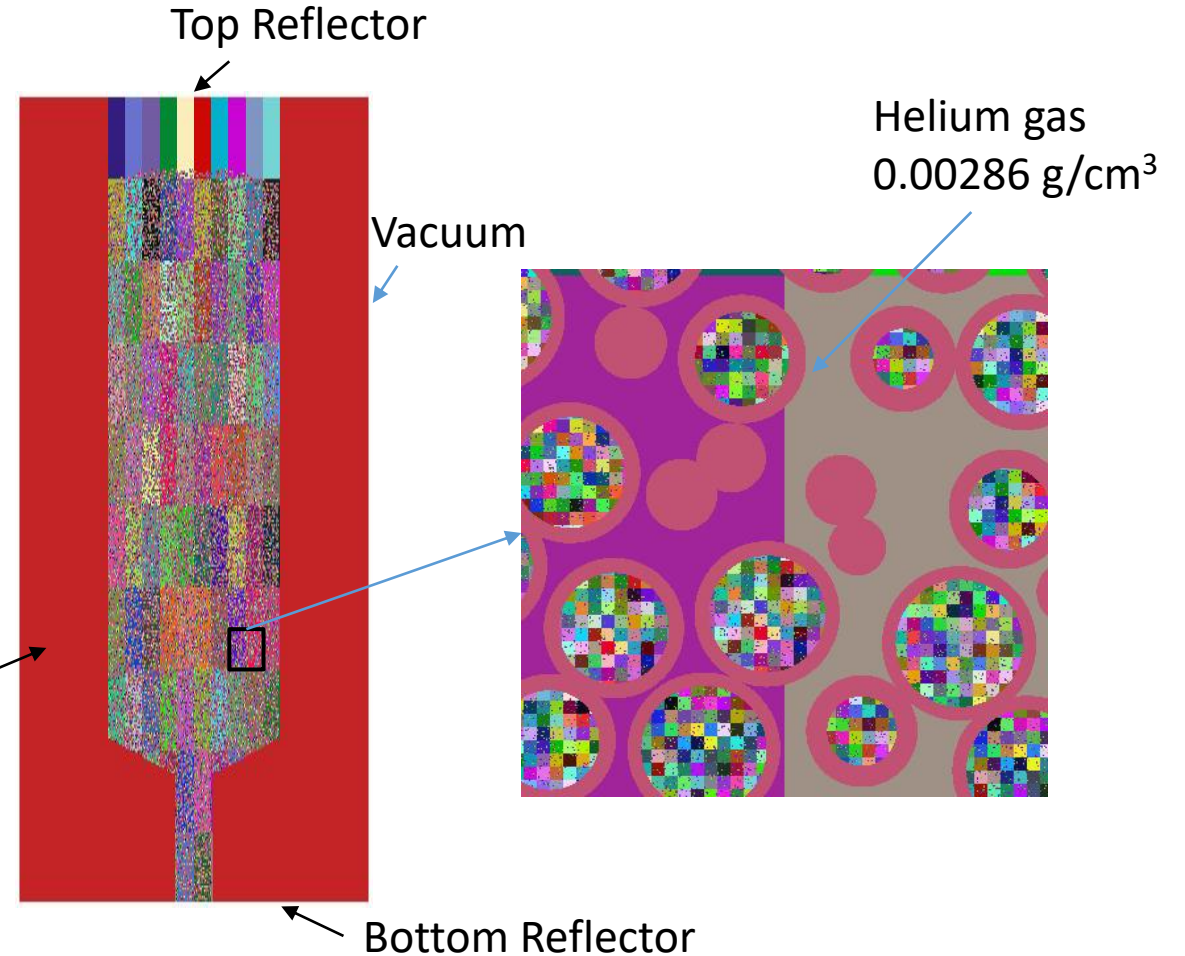
# OpenMC Preliminary Full Reactor Model

Read Pebbles position from .vtk in OpenMC



Modelling of reactor Core and RPV in OpenMC

RPV - dia 4.88 m  
(Graphite Material)



$k_{eff}$	Leakage fraction
$1.39160 \pm 0.00021$	$0.04780 \pm 0.00005$

# Acknowledgements

- Savannah Fitzwater and Ben Cipiti
  - Program managers
- Claudio Gariazzo, Yonggang Cui, Philip Gibbs, Donny Hartanto
  - External advisors
- Brandon Grogan, Jonathan Wing
  - INDEPTH and SCALE assistance
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